

**SCALE6.2 ORIGEN library
produced from JENDL/AD-2017**

Japan Atomic Energy Agency

Chikara Konno

Mami Kochiyama

Hirokazu Hayashi

Introduction and objective



- ❑ ORNL released the **SCALE6.2** code in 2016.
- ❑ The **ORIGEN** code in SCALE6.2 is **easier** to use and **more precise** than the ORIGEN-S code.
 - ORIGEN uses **one group** cross section data generated from a **specified neutron spectrum** and a **multigroup activation library** with the **COUPLE** code, not three group cross section data for ORIGEN-S.
- ❑ We expect that **ORIGEN in SCALE6.2 will be mainly used for activation calculations** in nuclear facility decommissioning.
- ❑ Thus we produced a **SCALE6.2 ORIGEN library** from JENDL Activation Cross Section File for Nuclear Decommissioning 2017 (**JENDL/AD-2017**) in order to popularize JENDL/AD-2017.

Processing Method -(1)



#3

- ❑ Code : **AMPX-6** in SCALE6.2.2
- ❑ Condition :
 - Group structure : **200 groups** (the same as one of libraries attached in SCALE6.2)
 - ✓ Assume transport calculations with MATXSLIB-J40 (199 groups)
 - ✓ The 200 group structure is the same as the 199 group structure except for its first group.
 - Temperature : **300 K**
 - ✓ Assume activation calculations for bio-shield concrete
 - Weighting function : **Maxwellian - 1/E - fission spectrum - 1/E** (above 10 MeV)
 - Infinite dilution

Processing Method -(2)



#4

□ AMPX-6 processing :

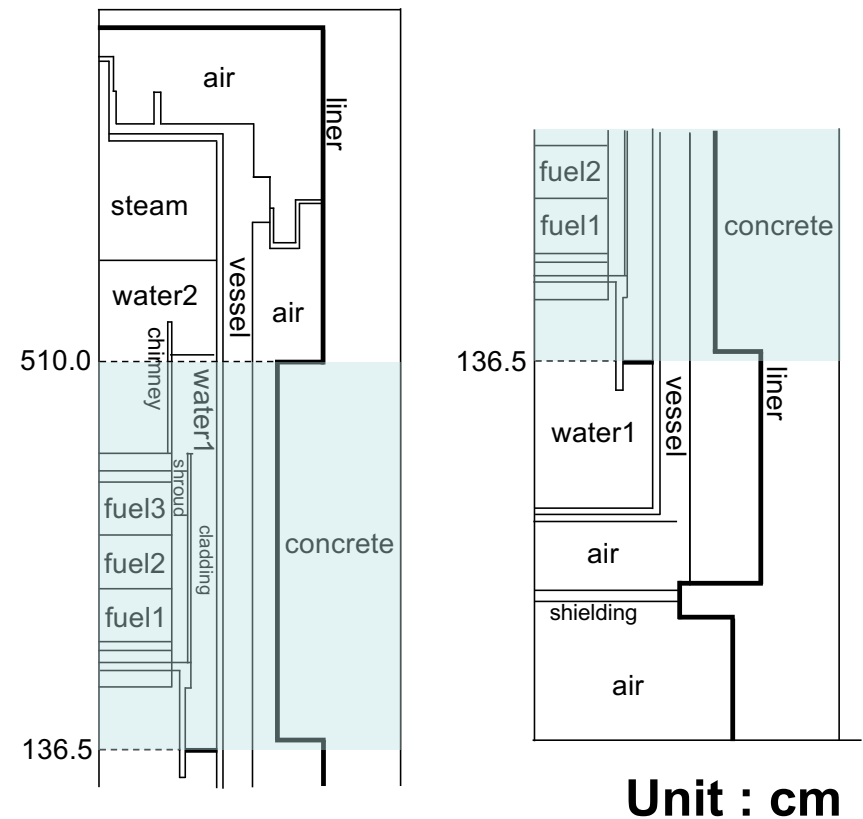
- **ExSite** code in AMPX-6 package produces a template input file for AMPX-6.
- We produce an ORIGEN library from each file of JENDL/AD-2017 with AMPX-6 and a modified template input and combine all the ORIGEN libraries to one ORIGEN library with AMPX-6.

□ Remarks on JENDL/AD-2017 :

- **Use 0K MF10 version** because 0K MF9 version is not processed.
- **Modify a metastable state level** based on the decay data of SCALE6.2 (Ex. ^{134m}Cs level : 3 in JENDL/AD-2017, 1 in decay data of SCALE6.2)
- **Remove unnecessary resonance data** because AMPX-6 considers them

Test with JPDR activation calculation -(1)

- We calculate **radioactive inventories in bio-shield concrete** of **JAEA JPDR** (Japan Power Demonstration Reactor).
- **Neutron spectra** inside the concrete are calculated with the **Sn DORT** code, a 199 group library including up-scattering data from **MATXSLIB-J40** and a simplified JPDR model.



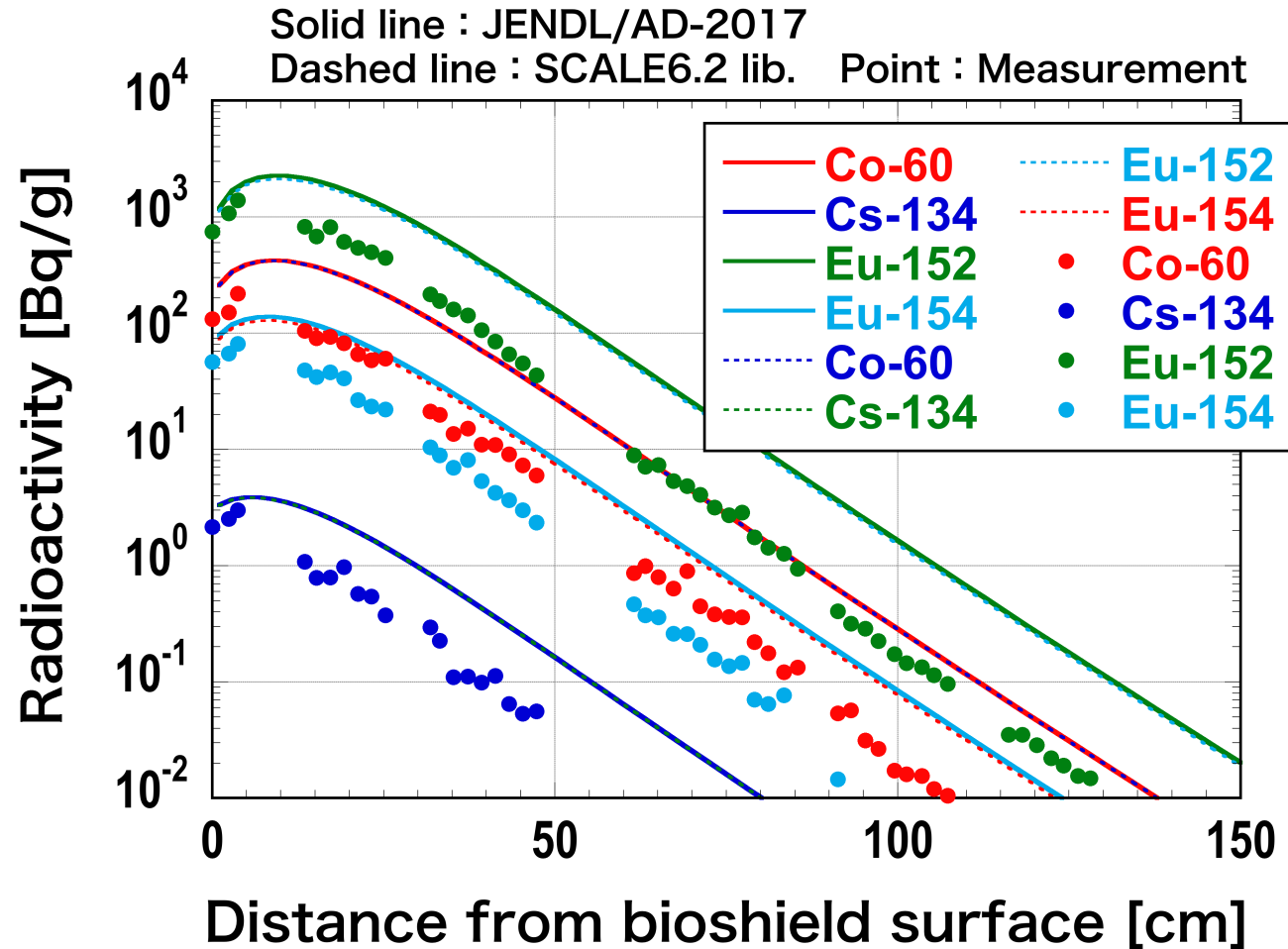
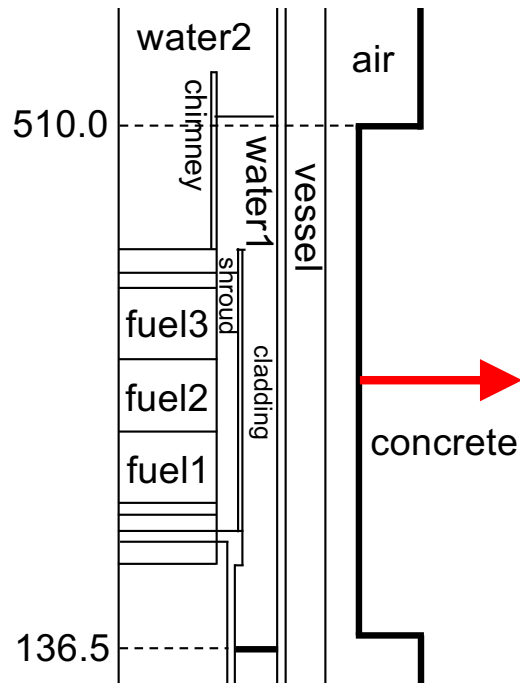
Region of from 136.5 cm to 510 cm (vertical) is modeled in DORT.

Test with JPDR activation calculation -(2)



- We calculate radioactive inventories in the concrete with **COUPLE** and **ORIGEN** in SCALE6.2.2 and calculated neutron spectra.
- Activation libraries :
 - the 200 group library produced from **JENDL/AD-2017**
 - the 200 group library **attached in SCALE6.2** (produced from JEFF-3.1/A by ORNL)
- 0.0 is added before 199 group neutron spectra because 200 group structure is the same as 199 group structure except for its first group.
- Time of 161 ORIGEN and COUPLE calculations is **4 minutes** (cf. 2.5 minutes for ORIGEN-S).

Test with JPDR activation calculation -(3)



Radioactivity distribution at Z=340cm (cooling : 15 years)

Calculation results with two libraries are almost the same.

→ The library from JENDL/AD-2017 has no problem!

Summary



- ❑ We produced a **200 group SCALE6.2 ORIGEN library** from **JENDL/AD-2017** with **AMPX-6**.
- ❑ **Radioactive inventories** in bio-shield concrete of JAEA **JPDR** were calculated with **ORIGEN** in SCALE6.2.2 for testing the library.
 - the 200 group library produced from JENDL/AD-2017
 - the 200 group library attached in SCALE6.2 (produced from JEFF-3.1/A by ORNL)
 - **Almost the same results**
 - **The produced library has no problem.**
- ❑ We will release the produced library.