

SCALE6.2 ORIGEN library produced from JENDL/AD-2017

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Oak Ridge National Laboratory released the SCALE6.2 code [1] in 2016 (the latest version is SCALE6.2.4). The ORIGEN code [1] in SCALE6.2 is completely different from the ORIGEN-S code [2] until SCALE6.0 [2].

1) ORIGEN uses one group cross section data generated from a specified neutron spectrum and a multigroup activation library with the COUPLE code [1], not three group cross section data with a typical neutron spectrum.

2) The input format of ORIGEN is easy to use and understand.

3) It is expected that the calculation accuracy improves because ORIGEN uses one group cross section data generated from neutron spectra in all calculation points.

4) The calculation time of ORIGEN including COUPLE is at most about twice of that of ORIGEN-S even for 200 groups.

We expect that ORIGEN in SCALE6.2 will be mainly used for activation calculations in nuclear facility decommissioning. Thus we produced a SCALE6.2 ORIGEN library from JENDL Activation Cross Section File for Nuclear Decommissioning 2017 (JENDL/AD-2017) [3] with the AMPX-6 [4] in order to popularize JENDL/AD-2017 widely. The processing conditions are as follows.

- Temperature : 300 K

- Group structure : 200 groups (the same as one of libraries attached in SCALE6.2)

- Weight function : Maxwell+1/E+Fission spectrum + 1/E (above 10 MeV)

- Infinite dilution

We tested the SCALE6.2 ORIGEN library of JENDL/AD-2017 with the JPDR decommissioning data [5], which demonstrated the library had no problem.

References

[1] ORNL, "SCALE: A Modular Code System for Performing Standardized Computer Analyses for Licensing Evaluation," ORNL/TM-2005/39 Version 6, Oak Ridge National Laboratory (2009).

[2] (Ed.) W.A. Wieselquist, R.A. Lefebvre, M.A. Jessee, "SCALE Code System," ORNL/TM-2005/39 Version 6.2.4, Oak Ridge National Laboratory (2020).

[3] <https://www.ndc.jaea.go.jp/ftpnd/jendl/jendl-ad-2017.html>

[4] D. Wiarda, M.E. Dunn, N.M. Greene, M.L. Williams, C. Celik, L.M. Petrie, "AMPX-6: A Modular Code System for Processing ENDF/B," ORNL/TM-2016/43, Oak Ridge National Laboratory (2016).

[5] T. Sukegawa, N. Sasamoto, K. Fujiki, "ACCURACY VERIFICATION FOR CALCULATION OF INVENTORY IN JPDR DUE TO NEUTRON ACTIVATION," INDC(JPN)-164, International Atomic Energy Agency (1993).

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